Attachment to the report prepared for the State Council (Gossovet) meeting on environmental issues chaired by Russian President Dmitry Medvedev on June 9, 2011.

Attachment 3

Plan of measures to enhance safety levels at nuclear power plants

| No. | Problem | Solution Approach | Timeframe | Executor organisation | | |
|-----|---|--|-----------|-----------------------------|--|--|
| | 1. Seismic performance and seismic strength | | | | | |
| 1.1 | Seismicity levels of a number of sites with NPP ¹ s are underrated | 1. Conduct a re-evaluation of seismicity of NPPs; | 2012 | State Corporation Rosatom, | | |
| | compared to the "General Seismic Zoning Maps of Russia" (OSR- | introduce amendments to supporting documentation | | OAO Concern | | |
| | 97), approved by the Russian Academy of Sciences. There are no | if needed. | | Rosenergoatom, Schmidt | | |
| | regulations determining the procedure for assessing seismicity of | 2. Provide regulatory framework for establishing the | | Institute of Physics of the | | |
| | sites of nuclear power plants; no body with authority to issue such | procedure of determining seismicity levels of sites of | | Earth (Russian Academy of | | |
| | assessments is established, either. | nuclear power plants and establish a body with | | Sciences) | | |
| | | authority to determine such levels. | | | | |
| 1.2 | Not all of the nuclear power plants have the option of automatic | Introduce a system of seismometric control, warning | 2012 | OAO Concern | | |
| | emergency reactor shutdown provided for earthquakes of a given | system and that of reactor emergency protection. | | Rosenergoatom | | |
| | intensity level. | | | | | |
| 1.4 | Seismic strength category is underestimated ² or not determined at | Assign seismic strength categories, as per NP-031- | 2012 | OAO Concern | | |
| | all for a number of NPP systems and equipment as per NP-031- | 01, too all NPP components, including construction | | Rosenergoatom | | |
| | 01^{3} . | elements. | | | | |
| 1.5 | Compliance with seismic strength requirements is not | Substantiate compliance with seismic strength | 2012 | OAO Concern | | |
| | substantiated for some of the safety-significant NPP equipment | requirements for safety-significant NPP equipment. | | Rosenergoatom | | |
| | (for instance, mounting fixtures of the separator drum at Bilibino | | | | | |
| | NPP). In a number of cases, existing calculations serving as a | | | | | |
| | basis for seismic strength assessments are not supported by | | | | | |
| | experimental studies. | | | | | |
| | 2. Management of spent nuclear fuel (SNF) | | | | | |

¹ Nuclear power plants. – Translator.

² "Overestimated" is possibly what is meant, as seismic strength of particular systems is presumed to be less reliable than the seismic resistance class they are currently assigned. – Translator.

³ For "Norms of Designing Earthquake-Resistant Nuclear Power Plants," introduced by the former State Agency for Nuclear Supervision (now, Federal Service for Environmental, Technological and Nuclear Oversight of Russia) in 2002. – Translator.

| provided for Beloyarsk NPP. No storage facility is available for long-term storage of SNF generated at Reactor Units 1, 2 of Beloyarsk NPP. On-site SNF storage facilities (both cooling ponds and stand-alone SNF storage facilities) are in a critical condition. 2.2 The final stage of SNF management has not been determined for SNF generated at Bilibino NPP. Develop an SNF management programme for SNF generated by Bilibino NPP. Corporate A Bilibino NPP. Corporate A Roser State Peloyarsk NPP for long-term storage or further reprocessing (regeneration). Develop an SNF management programme for SNF generated by Bilibino NPP. Corporate A Bilibino NPP. | O Concern energoatom, ee Corporation Rosatom O Concern energoatom, State eporation Rosatom O Concern |
|--|--|
| long-term storage of SNF generated at Reactor Units 1, 2 of Beloyarsk NPP. On-site SNF storage facilities (both cooling ponds and stand-alone SNF storage facilities) are in a critical condition. 2.2 The final stage of SNF management has not been determined for SNF generated at Bilibino NPP. Develop an SNF management programme for SNF generated by Bilibino NPP. Corporate SNF generated at Reactor Units 1, 2 of Beloyarsk NPP for long-term storage or further reprocessing (regeneration). | O Concern senergoatom, State poration Rosatom |
| Beloyarsk NPP. On-site SNF storage facilities (both cooling ponds and stand-alone SNF storage facilities) are in a critical condition. 2.2 The final stage of SNF management has not been determined for SNF generated at Bilibino NPP. Develop an SNF management programme for SNF generated by Bilibino NPP. Roser Corporate. | O Concern senergoatom, State poration Rosatom |
| and stand-alone SNF storage facilities) are in a critical condition. 2.2 The final stage of SNF management has not been determined for SNF generated at Bilibino NPP. Develop an SNF management programme for SNF generated by Bilibino NPP. Corporate to the final stage of SNF management programme for SNF generated by Bilibino NPP. Corporate to the final stage of SNF management programme for SNF generated by Bilibino NPP. | energoatom, State poration Rosatom |
| 2.2 The final stage of SNF management has not been determined for SNF generated at Bilibino NPP. Develop an SNF management programme for SNF generated by Bilibino NPP. OAO generated by Bilibino NPP. Corporate | energoatom, State poration Rosatom |
| SNF generated at Bilibino NPP. generated by Bilibino NPP. Corpo | energoatom, State poration Rosatom |
| Corpo | poration Rosatom |
| | |
| | O Concern |
| | , |
| | senergoatom, |
| | te Corporation Rosatom |
| amounts far in excess of design-basis values. The accumulation of 2. Improve the schedule of construction of a spent 2011 | |
| the amounts of SNF in the on-site cooling ponds is only taking fuel assembly dismantlement shop at Smolensk NPP. | , |
| place at the expense of reducing the grid spacing that serves to | , |
| separate spent fuel assemblies in the cooling ponds, which leads to | , |
| impaired subcriticality of the SNF. | |
| 3. Management of radioactive waste | |
| 3.1 Classification of radioactive waste as per the sanitary rules in force Bring the classification of LRW and GRW ⁶ as per 2011 Chief | • |
| | pector of the Russian |
| | eration. |
| Thus, the introduction of the criteria stipulated in OSPORB- to intervention levels (10IL) ⁸ and the maximum | , |
| 99/2010 reduces significantly radiation safety for the personnel allowable level of airborne activity concentration for | , |
| and the population when managing radioactive waste at an NPP. the population (AACpop ⁹). | , |
| | |
| | O Concern |
| and SWSFs ¹⁰) are filled to 60 percent. The LWSFs of Smolensk radioactive waste storage and reprocessing Roser | senergoatom, State |
| and Leningrad NPPs, as well as the SWSFs of Kursk and complexes at Leningrad, Smolensk, and Kursk Corpo | poration Rosatom |

For "spent fuel assembly." – Translator.
 For Main Sanitary Rules of Assuring Radiation Safety-99/2010, of 2010; Sanitary Rules for Designing and Operating Nuclear Power Plants-03, of 2003; and Sanitary Rules of

Management of Radioactive Waste-2002, of 2002, respectively. – Translator.

6 For Liquid Radioactive Waste and Gaseous Radioactive Waste, respectively. – Translator.

7 For Norms of Radiation Safety-99/2009, of 2009. – Translator.

8 Levels of avertable dose at which a specific protective action or remedial action is taken in an emergency exposure situation or a chronic exposure situation (from the IAEA). Nuclear Safety and Security Glossary). – Translator.

9 For Allowable Activity Concentration for the population. – Translator.

¹⁰ For Liquid Waste Storage Facilities and Solid Waste Storage Facilities, respectively. – Translator.

| | Leningrad NPPs are filled to more than 85 percent. No NPP is at | NPPs. | | | | |
|-----|--|--|------|----------------------|--|--|
| | present equipped with a full-cycle LRW reprocessing complex. 11 | | | | | |
| 3.3 | No unified scientific and technological approach exists for | Develop a long-term radioactive waste management | 2012 | OAO Concern | | |
| | radioactive waste management at new NPP reactor units. | strategy for new NPP reactor units. | | Rosenergoatom, State | | |
| | - | | | Corporation Rosatom | | |
| | 4. Structural integ | rity of building components and equipment | | • | | |
| 4.1 | Reactor buildings of a number of reactor units (for instance, at | Run an analysis and develop a plan of measures to | 2012 | OAO Concern | | |
| | Balakovo NPP and Kalinin NPP) show progressing subsidence | stabilise the settlements and building tilt. | | Rosenergoatom | | |
| | and differential settlements, as well as progressing tilt, which may | _ | | | | |
| | lead to their structural failure. | | | | | |
| 4.2 | A bend in the foundation plate of the SFSF ¹² building at Kursk | Substantiate safety of further operation of the SFSF | 2014 | OAO Concern | | |
| | NPP, revealed in the course of geodetic observations, can lead to | building's foundation plate. | | Rosenergoatom | | |
| | the building's failure. | 1 | | | | |
| 4.3 | Intragranular stress corrosion cracking in welding seams of DN | 1. Complete the implementation of "Programme of | 2012 | OAO Concern | | |
| | 300 (Du 300) austenitic pipelines and joint seams at the bottoms of | works No. NPP RBMK PRG-19 (04-03)2010 to | | Rosenergoatom | | |
| | distributing group headers of the multiple forced circulation circuit | complete the task of solving the issue of welding | | | | |
| | of RBMK-1000 reactor plants: The mechanism of development of | seams of Du 300 austenitic pipelines of RBMK-1000 | | | | |
| | longitudinal and transverse cracks in the deposited weld metal in | reactor plants." | | | | |
| | the welding seams has not been established; the ultrasound control | 2. Complete the introduction of the comprehensive | 2011 | | | |
| | methods that serve to detect such cracks are not sufficiently | coolant leak control system (ACLDS ¹³). | | | | |
| | developed. | • • • • | | | | |
| 4.4 | Structural strength (impact resistance) of construction elements of | 1.Carry out substantiation of necessary measures to | 2013 | OAO Concern | | |
| | most NPPs does not meet the requirements of the regulations in | ensure structural strength (impact resistance) of NPP | | Rosenergoatom | | |
| | force with respect to loads occurring as a result of extreme | construction elements against loads occurring as a | | | | |
| | environmental impacts. | result of extreme environmental impacts. | | | | |
| | - | 2. Upon analysis completion, implement the | | | | |
| | | measures necessitated by the results of said analysis. | 2015 | | | |
| | 5. Hydrogen explosion safety. | | | | | |
| 5.1 | Hydrogen concentration control systems and systems and | v c i | 2012 | OAO Concern | | |
| 3.1 | components ensuring hydrogen explosion safety do not meet the | conformity with NP-040-02. | 2012 | Rosenergoatom | | |
| | requirements of the Rules of Hydrogen Explosion Safety | Comornity with Ni -0-10-02. | | Roschergoatom | | |
| | Assurance at Nuclear Power Plants (NP-040-02). | | | | | |
| | Assurance at inductal rower rialits (inf-040-02). | | | | | |

The original Russian text is unclear as to whether "none of the mentioned NPPs" is meant here or none of Russia's ten NPPs in operation. – Translator.

Spent Fuel Storage Facility. – Translator.

Translator. – Translator.

| (1 | | structure, management, and safety culture. | 2011 | |
|-----|---|--|--|---|
| 6.1 | No criteria are developed for the assessment of the necessary financial and human resources that the Operator Organisation (OO) must have at its disposal both as part of its head office staff and in branch organisations in order to ensure NPP safety. | Establish the criteria needed to determine the baseline staffing requirements with respect to the necessary number of production and engineering personnel and their qualitative composition in order to fulfil the Operator Organisation's functions both | 2011 | State Corporation Rosatom, OAO Concern Rosenergoatom |
| | | at the head office and in branch organisations. | | |
| 6.2 | The OO maintains no efficient cumulative records on NPP operational experience, which results in: - low quality of investigations into operational disturbances at NPPs by the OO's commissions (in seven instances throughout 2010, Rostekhnadzor ¹⁴ was forced to return investigation reports for additional investigation), - low quality of remedial measures developed by the OO, - operational violations occurring repeatedly for the same reason, - underreporting the number of defects, system or equipment failures, violations or operational disturbances. | Develop measures to improve the quality of investigations. Carry out an analysis and develop steps to improve the quality of remedial measures. | 2011 | OAO Concern Rosenergoatom |
| 6.3 | Reorganisation and casualisation of NPP maintenance staff leads to outflow of highly skilled personnel. | Develop and implement staffing policy measures at the OO where it concerns retaining highly qualified personnel. | 2011 | OAO Concern Rosenergoatom |
| 6.4 | Shortage of inspectorial staff at Rostekhnadzor results in a reduced level of efficiency of maintaining supervision over nuclear and radiation safety at NPPs. | For the Government of the Russian Federation, establish and implement a financing mechanism needed to ensure efficient control over nuclear and radiation safety. | To be determined by the Government of the Russian Federation | Federal Service for Environmental, Technological and Nuclear Oversight of Russia |
| 6.5 | At a number of NPPs, full cover is not provided in protective shelters to accommodate the largest number of workers on shift in case of an emergency. | Ensure that full cover is provided in protective shelters at all NPPs for the largest team on shift in case of an emergency. | 2013 | OAO Concern Rosenergoatom |
| 6.6 | The more than 20-year-long operation of four reactor units (Units 1, 2 of Novovoronezh NPP and Units 1, 2 of Beloyarsk NPP) in full shutdown mode results in an absence of solutions to the issues of their decommissioning. No applications for licenses to be issued | Ensure that applications for decommissioning licenses are filed with Rostekhnadzor for Reactor Units 1, 2 of Novovoronezh NPP and Reactor Units 1, 2 of Beloyarsk NPP. | 2011 | OAO Concern Rosenergoatom |

_

¹⁴ Or Federal Service for Environmental, Technological and Nuclear Oversight of Russia. – Translator.

| for the decommissioning of these reactor units have as yet been | | | |
|--|---|---|---|
| filed with Rostekhnadzor. | | | |
| 7. Performance | ability of safety-significant components. | | |
| No systemic analysis is done with respect to the safety of equipment that serves to perform safety functions. Its reliability is not ascertained for conditions arising due to external impacts or during design-basis accidents. | Develop a branch-wide programme and branch-wide methods and procedures for analysing the protectedness of equipment that serves to perform safety functions against general failures and for ascertaining its qualification. Perform equipment analysis in accordance with the methods developed. Implement a complex of necessary measures as required by the results of said analysis. | 2012 2013 2014 | OAO Concern Rosenergoatom |
| Flaws and defects in the design of the actuators of control rod clusters (CRC) of RBMK-1000 reactor plants could lead to accidents. | actuators, present a plan of action to enhance operational reliability of CRCs (based on proposals detailing ways to improve the CRC design), | 2011 | State Corporation Rosatom, OAO Concern Rosenergoatom |
| NPP project documentation lists an unfoundedly lower safety class for a number of systems and equipment items, as per NP-001-97 ¹⁵ . For instance, assigning current/voltage transformers to Safety Class 4 results in an unsatisfactory quality of the equipment used to monitor the functioning of NPP electrical power output systems (failures experienced by components of these systems cause reactor units to switch off the grid and trigger safety system response). | Conduct an analysis based on principles of conservative approach with the purpose of verifying that the adopted classification of systems and equipment is in conformity with the requirements set forth by relevant norms and rules. Conduct reclassification if needed. | 2012 | State Corporation Rosatom, OAO AEP (Atomenergoproyekt, Moscow), OAO NIAEP (Nizhny Novgorod Engineering Company Atomenergoproyekt), OAO SPbAEP (Atomenergoproyekt, St. Petersburg) |
| No framework of regulatory and engineering provisions exists for the creation, verification, validation, qualification, and certification of automated process control systems (APCS). Deliveries of APCS equipment to NPP reactor units under construction or renovation are done absent of full-scale and comprehensive tests conducted on bench or test models. | Develop regulatory norms to enable creation, verification, validation, qualification, and certification of APCSs. | 2013 | State Corporation Rosatom, OAO Concern Rosenergoatom |
| | No systemic analysis is done with respect to the safety of equipment that serves to perform safety functions. Its reliability is not ascertained for conditions arising due to external impacts or during design-basis accidents. Flaws and defects in the design of the actuators of control rod clusters (CRC) of RBMK-1000 reactor plants could lead to accidents. NPP project documentation lists an unfoundedly lower safety class for a number of systems and equipment items, as per NP-001-97 ¹⁵ . For instance, assigning current/voltage transformers to Safety Class 4 results in an unsatisfactory quality of the equipment used to monitor the functioning of NPP electrical power output systems (failures experienced by components of these systems cause reactor units to switch off the grid and trigger safety system response). No framework of regulatory and engineering provisions exists for the creation, verification, validation, qualification, and certification of automated process control systems (APCS). Deliveries of APCS equipment to NPP reactor units under construction or renovation are done absent of full-scale and | T. Performance ability of safety-significant components. No systemic analysis is done with respect to the safety of equipment that serves to perform safety functions. Its reliability is not ascertained for conditions arising due to external impacts or during design-basis accidents. In Develop a branch-wide programme and branch-wide methods and procedures for analysing the protectedness of equipment that serves to perform safety functions against general failures and for ascertaining its qualification. Perform equipment analysis in accordance with the methods developed. 3. Implement a complex of necessary measures as required by the results of said analysis. Upon running an analysis of the design of CRC actuators, present a plan of action to enhance operational reliability of CRCs (based on proposals detailing ways to improve the CRC design), complete with specific implementation time frames. NPP project documentation lists an unfoundedly lower safety class for a number of systems and equipment items, as per NP-001-97 ¹⁵ . For instance, assigning current/voltage transformers to Safety class 4 results in an unsatisfactory quality of the equipment used to monitor the functioning of NPP electrical power output systems (failures experienced by components of these systems cause reactor units to switch off the grid and trigger safety system response). No framework of regulatory and engineering provisions exists for the creation, verification, validation, qualification, and certification of automated process control systems (APCS). Deliveries of APCS equipment to NPP reactor units under construction or renovation are done absent of full-scale and | T. Performance No systemic analysis is done with respect to the safety of equipment that serves to perform safety functions. Its reliability is not ascertained for conditions arising due to external impacts or during design-basis accidents. Elaws and defects in the design of the actuators of control rod clusters (CRC) of RBMK-1000 reactor plants could lead to accidents. Flaws and defects in the design of the actuators of control rod clusters (CRC) of RBMK-1000 reactor plants could lead to accidents. NPP project documentation lists an unfoundedly lower safety class for a number of systems and equipment items, as per NP-001-97 ¹⁵ . For instance, assigning current/voltage transformers to Safety Class 4 results in an unsatisfactory quality of the equipment used to monitor the functioning of NPP electrical power output systems (failures experienced by components of these systems cause reactor units to switch off the grid and trigger safety system response). No framework of regulatory and engineering provisions exists for the creation, verification, validation, qualification, and certification of automated process control systems (APCS). Deliveries of APCS equipment to NPP reactor units under construction or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification or renovation are done absent of full-scale and certification of accertain programme and branch-wide methods dequipment that serves to perf |

¹⁵ For General Standards and Rules of Assuring Safety at Nuclear Power Plants-001-97, of 1997. – Translator.

| 8.1 | Terms and conditions of safe operation are not substantiated in full | As per PSA ¹⁶ results, carry out substantiation of | 2013 | OAO Concern |
|-----|--|---|------|---------------|
| 0.1 | where they pertain to: | terms and conditions of safe operation. | 2013 | Rosenergoatom |
| | - requirements set to the minimum amount of operational | terms and conditions of said operation. | | resemengoutem |
| | equipment in use, | | | |
| | - the allowable time limit set for taking equipment out of | | | |
| | operational condition, | | | |
| | - periodicity of audits to verify operational condition of given | | | |
| | equipment. | | | |
| 8.2 | The list of accident scenarios provided for by ILA and RUZA ¹⁷ is | 1. Conduct, for all NPPs, an analysis to examine the | 2012 | OAO Concern |
| | incomplete: Unprovided for are accidents occurring during fuel | scope of coverage provided by ILA and RUZA for | | Rosenergoatom |
| | management, accidents occurring on reactor units in shutdown, | accidents occurring on reactor units in shutdown, | | |
| | fires, floods, external impacts occurring as a result of | accidents occurring during nuclear fuel management | | |
| | environmental or technogenic causes. | and storage, fires, floods, and external impacts. | | |
| | Full analysis is not provided for fires and floods resulting from | 2. Improve ILA, RUZA as per analysis results. | 2013 | |
| | internal causes as well as external impacts of environmental or | | | |
| | technogenic nature. | | | |
| | No guidelines exist for management of severe accidents. | | | |
| 8.3 | No analyses have been performed for multi-unit NPPs (excluding | 1. Conduct an analysis of possible impact of | 2014 | OAO Concern |
| | Kola NPP) with a view to examine the impact of technogenic | violations of normal operation (accidents) on | | Rosenergoatom |
| | events (violations of normal operation) occurring at one unit on | adjacent units of multi-unit NPPs (fires, floods, | | |
| | the safety of adjacent units. | objects in flight, accidental explosions, release of | | |
| | Neither have analyses been performed to examine the impact on | hazardous substances, radiation accidents). | | |
| | NPP safety of violations involving non-unit-specific equipment | 2. Conduct an analysis of possible impact on NPP | 2014 | |
| | (chemical facilities and equipment, vessels under pressure, | safety of violations in the operation of non-unit- | | |
| | hydroengineering structures). | specific equipment. | | |
| 8.4 | No substantiation is provided for the capability of reactor units of | 1. Conduct BDBA ²⁰ analysis for accidents involving: | 2015 | OAO Concern |
| | nuclear power plants to maintain safe condition during prolonged | a) prolonged (up to several days) outage of CP | | Rosenergoatom |
| | periods of time (up to several days) during accidents caused by: | cooling systems, | | |
| | - loss of CP ¹⁸ cooling systems, | b) loss of power for internal needs compounded by | | |
| | - loss of power at the reactor unit compounded by failure of diesel | DG ²¹ failure, | | |
| | generators, | c) failure of service water supply system on the side | | |
| | - failure of service water supply system on the side of the essential | of the essential power consumer. | | |
| | power consumer. ¹⁹ | 2. As per analysis results, implement necessary | 2017 | |

For Probabilistic Safety Assessment. – Translator.

17 For Accident Liquidation Guidelines and Beyond-Design-Basis Accident Management Manual, respectively. – Translator.

18 For "cooling ponds." – Translator.

| | | measures to provide for management of the above BDBAs. | | |
|-----|--|---|------|------------------------------|
| 8.5 | No analysis has been done of the impact on the safety of Novovoronezh NPP of shockwaves following a burst accident at hydropower installations, including Voronezh and Matyra Waterworks. The analysis done of the impact on the safety of Balakovo NPP of an accident at Kuibyshev Hydropower Plant is in need of being checked for currency. | 1 | 2015 | OAO Concern Rosenergoatom |
| 8.6 | For NPP units operating first-generation VVER-440 reactors (Units 3, 4 of Novovoronezh NPP, Units 1, 2 of Kola NPP): - the working efficiency of the jet-vortex condenser (JVC) in the conditions of a beyond-design-basis accident involving a rupture of the DN 500 (DU 500) primary circuit pipeline lacks proper substantiation. The operational efficiency of the JVC is not substantiated for conditions of external impacts; - no substantiation is provided for the sufficient cooling of the standby boric acid solution tank for accident conditions involving the largest-scale leak of the primary circuit provided for in the design, which could lead to an associated failure of the pumps of the emergency core cooling system; - the substantiation of application of the Leak Before Failure ²² concept is insufficient. | 1. Conduct an analysis of a beyond-design-basis accident involving a rupture of the DN 500 (DU 500) MCP ²³ that would take into account the possibility of pressure in the sealed enclosures exceeding maximum allowable values; as per analysis results, assess the sufficiency of technological and organisational measures available for accident management, provide substantiation for operational reliability of the JVC in conditions of external impacts. 2. Replace emergency feedwater pumps (ECPS ²⁴ pumps) with pumps capable of operating on water with a temperature of up to 100° Celsius. 3. Improve the Leak Before Failure concept. | 2015 | OAO Concern Rosenergoatom |

¹⁹ Or sites of priority uninterrupted power supply "where outage in power delivery may entail a threat to human lives, threat to national security, considerable material damage, disruption of complex technological processes, and disruption in the operation of especially important elements of public services, communication links, or television," as per the

Russian Ministry of Energy's Regulations for the Design and Construction of Electrical Installations of 2002. – Translator.

20 For "beyond-design-basis accident." – Translator.

21 For "diesel generator." – Translator.

22 Leak Before Failure implies that the design and selection of construction materials rule out the complete destruction of a component that has suffered a through crack without the initial development of a stable leak that can be detected well before the crack reaches critical values with respect to structural stability. – Translator.

For Main Circulation Pipeline. – Translator.
 For Emergency Core Protection System. – Translator.